

Profiles Control and Stability in Thermonuclear Fusion: Some Issues for ITER

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Abundant fuels:

- Deuterium: from all forms of water, for millions of years.
- Tritium: not natural, bred from Lithium.
- Lithium: from earth's crust, if all the world's electricity provided by fusion, known reserves would last for at least one thousand years.

High energetic gain:

- 10 g of D (500 l of water) and 15 g of T (30 g of L_j) \Rightarrow lifetime electricity needs of an average person in an industrialized country.

Inherent safety:

- Very small amount of D & T and difficult operating conditions → any deviation results in a rapid cooling of the plasma and its termination.
- No circumstances in which the fusion reaction can proceed into an uncontrollable or critical condition.

Environmental advantages:

- No 'greenhouse' gases (no global warming contribution).
- Rapid decay of the structure radioactivity and the time span before it can be re-used and handled can be minimised (to around 50 years).
- No radioactive 'waste' from the fusion reaction itself.
- The byproduct is H_e - an inert and harmless gas.

"Control in fusion: towards ITER"

E. Witrant and S. Brémond

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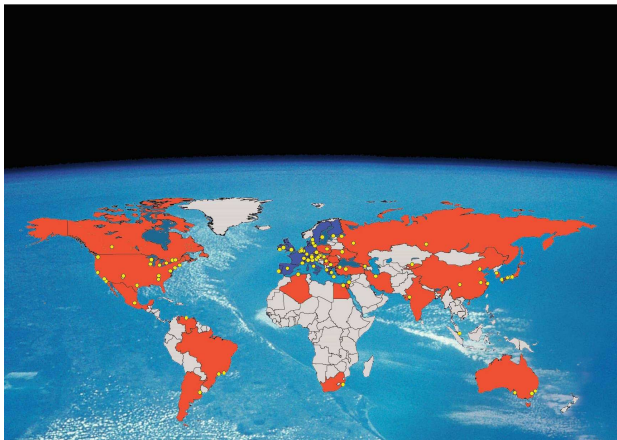
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Fusion and fission: exo-energetic processes

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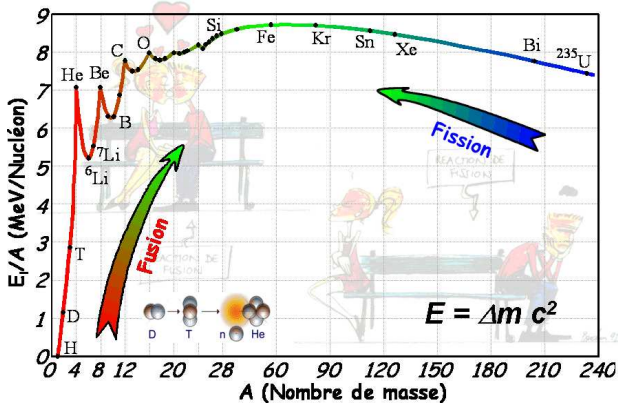
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Fusion vs. fission

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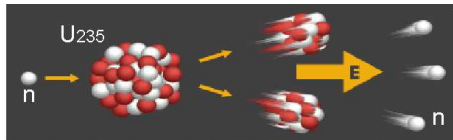
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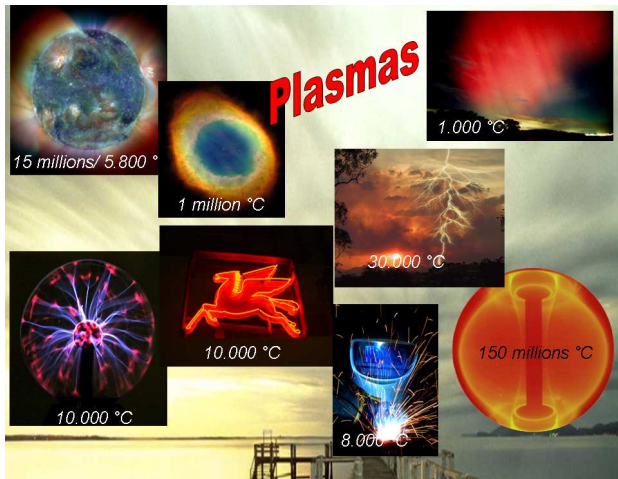
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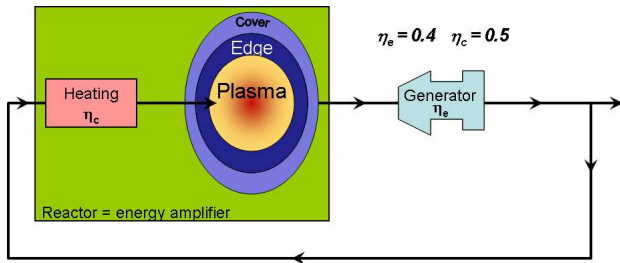
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Controlled fusion: Tokamaks



Principle of a fusion reactor



Performances:

- Energy confinement time
- Plasma energy
- Fraction of output power used to heat the plasma

Magnetic confinement: close the field lines to form a torus

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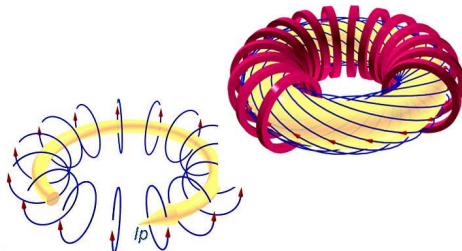
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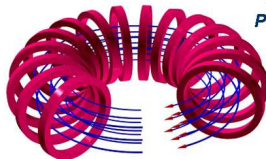
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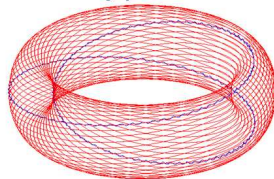
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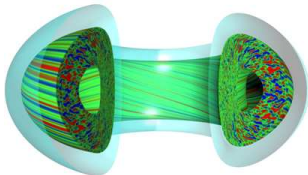
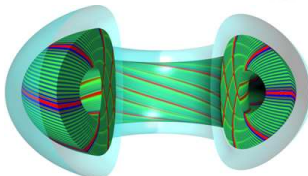
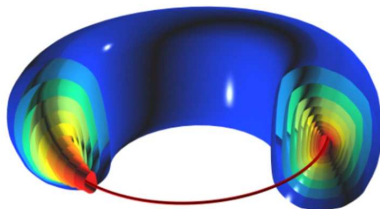
Poloidal field induced by I_p



Toroidal field



Plasma dynamics: isoflux surfaces . . . and turbulences!



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Plasma Physics Issues:

- MHD Stability
- Heat Confinement
- Steady State Operation
- Control of Plasma Purity
- Exploration of the new physics with a dominant α -particles plasma self heating

Tokamak automation

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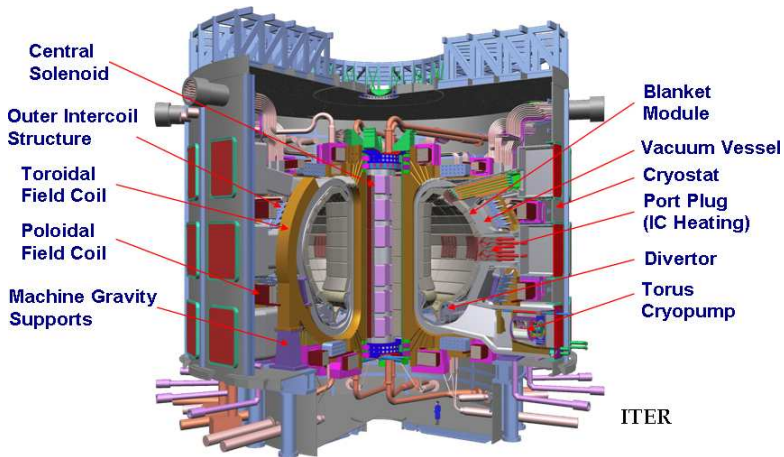
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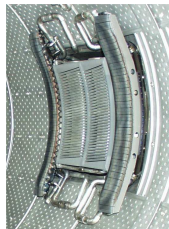
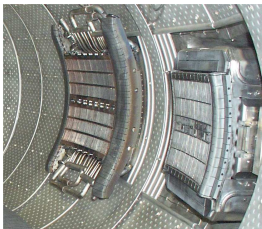
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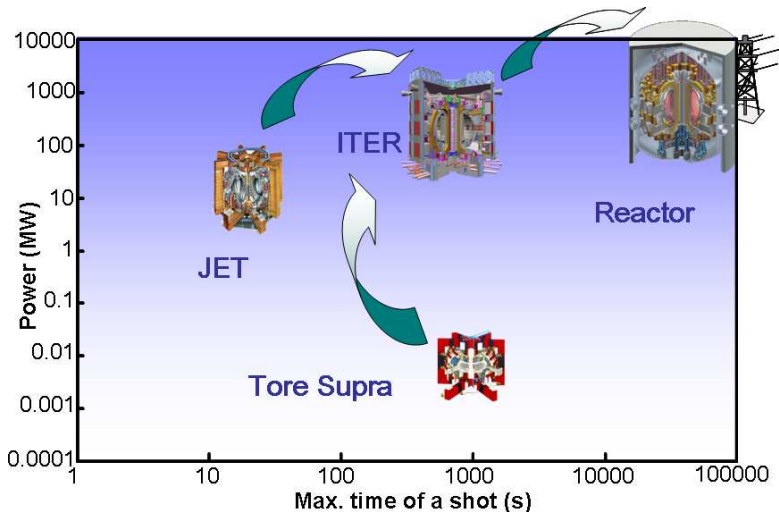


Radio frequency antennas (i.e. Tore Supra):

- Ion cyclotron radio heating (ICRH) - 40-80 Mhz (2 antennas, 10 MW)
- Lower hybrid (LH) 3.7 GHz (2 antennas, 16 klystrons, 6.2 MW)
- Electron cyclotron current drive (ECCD) - 118 Ghz (1 antenna, 2 gyrotrons, 0.8 MW)



Why ITER?



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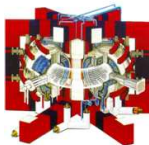
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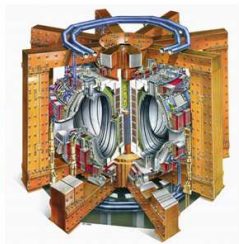
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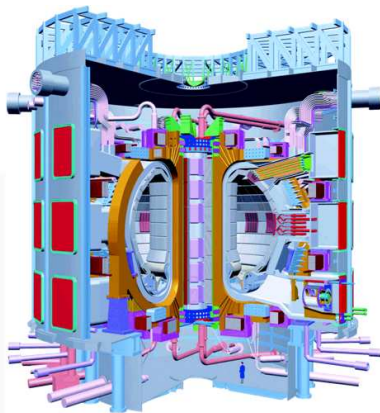
Some numbers ...



V_{plasma} 25 m³
 P_{fusion} ~0
 T_{plasma} ~400 s



V_{plasma} 80 m³
 P_{fusion} ~16 MW
 T_{plasma} ~30 s



V_{plasma} 830 m³
 P_{fusion} ~500 MW
 T_{plasma} ~400 s

Programmatic:

- Demonstrate the scientific and technological feasibility of fusion energy for peaceful purposes.

Technical:

- Demonstrate extended burn of DT plasmas, with steady state as the ultimate goal.
- Integrate and test all essential fusion power reactor technologies and components.
- Demonstrate safety and environmental acceptability of fusion.

Profiles control: modeling and feedback in Tore Supra

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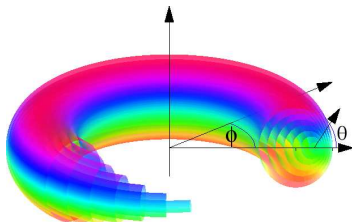
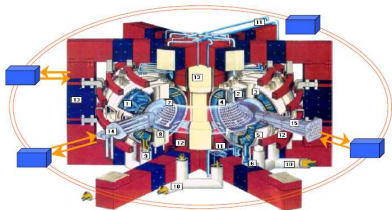
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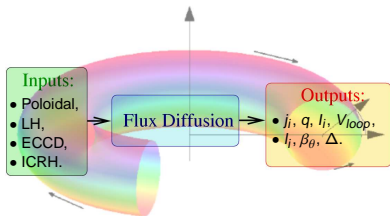
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- Advanced plasma confinement schemes \Rightarrow current density and temperature profiles control;
 - Focus on the magnetic flux dynamics and use experimental measurements;
 - Include bootstrap effect, poloidal, ECCD, ICRH, LH;
- \Rightarrow representative RT model of plasma physics for control synthesis on Tore Supra.

Flux diffusion dynamics



Hypotheses:

- cylindrical coordinates (neglect GSS),
- neglect diamagnetic effect,

System dynamics [Blum'89, Brégeon & al'98]:

$$\frac{\partial \psi}{\partial t}(x, t) = \eta_{\parallel}(x, t) \left[\frac{1}{\mu_0 a^2} \frac{\partial^2 \psi}{\partial x^2} + \frac{1}{\mu_0 a^2 x} \frac{\partial \psi}{\partial x} + R_0 j_{bs}(x, t) + R_0 j_{ni}(x, t) \right]$$

$$j_{\phi}(x, t) = -\frac{1}{\mu_0 R_0 a^2 x} \frac{\partial}{\partial x} \left[x \frac{\partial \psi}{\partial x} \right]$$

with $\psi'(0, t) = 0$, $\psi'(1, t) = f(I_p)$ or $\dot{\psi}(1, t) = f(V_{loop})$ and IC.

Time constant

Resistivity, temperature and density profiles

- Resistivity [Hirshman'77]:

$$\eta_{\parallel}(x, t) = f(T_e, n_e, \bar{Z}) \propto T_e^{-3/2}(n_e, \bar{Z})$$

- Temperature:
 - Profile shape determination \rightarrow parameter dependant identification of nonlinear distributed systems:
 - Grey-box modeling,
 - 3-hidden layers approach: spatial distribution, steady-state and transient behavior,
 - Stochastic descent method with direct differentiation.
 - Predictive mode: identified shape parameters + amplitude/[ITER'99]
 - Direct measurement
- Density - average or interferometer:

$$n_e(x, t) = n_{e0}(t)(1 - x^{\gamma_n}) \text{ and } n_{e0}(t) = \frac{\gamma_n + 1}{\gamma_n} \bar{n}_e(t).$$

Nonlinear component

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$$\frac{\partial \psi}{\partial t}(x, t) = \eta_{//}(x, t) \left[\frac{1}{\mu_0 a^2} \frac{\partial^2 \psi}{\partial x^2} + \frac{1}{\mu_0 a^2 x} \frac{\partial \psi}{\partial x} + R_0 j_{bs}(x, t) + R_0 j_{ni}(x, t) \right]$$

Bootstrap effect:

- Self induced current,
- Nonlinear component [Hirshman'88]:

$$j_{bs}(x, t) = \frac{eR_0}{\partial \psi / \partial x} \left\{ (A_1 - A_2) n_e \frac{\partial T_e}{\partial x} + A_1 T_e \frac{\partial n_e}{\partial x} + A_1 (1 - \alpha_i) n_i \frac{\partial T_i}{\partial x} + A_1 T_i \frac{\partial n_i}{\partial x} \right\}$$

with $n_i(x, t) \approx \frac{7 - \bar{Z}(t)}{6} n_e(x, t)$ and $T_i(x, t) \approx \alpha_{Ti}(t) T_e(x, t)$.

Controlled inputs

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- BC input from the poloidal coil:

$$V_c = R_c I_c + L_c \dot{I}_c + M \dot{I}_p$$
$$V_{loop} = M \dot{I}_c - \frac{1}{I_p} \frac{\partial}{\partial t} \left[\frac{L_p I_p^2}{2} \right] = \dot{\psi}_a$$

Local control loop: set V_c according to $I_{p,ref}$ or $V_{loop,ref}$.

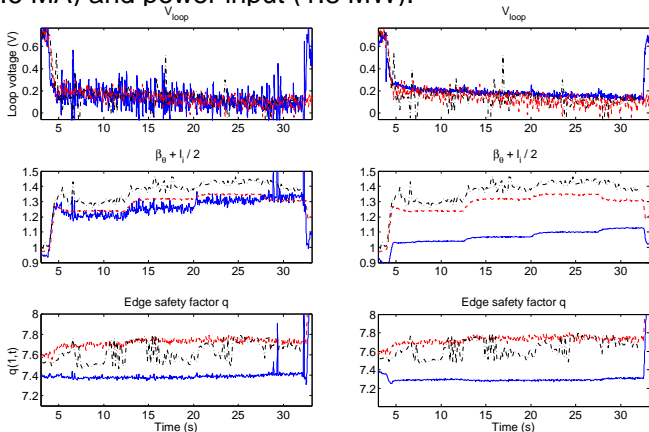
- Distributed current deposit from the antennas (LH, ECCD)
- **Engineering approach** with gaussian approximation:

$$j_{ni}(x, t) = \vartheta(t) e^{-(\mu(t)-x)^2/2\sigma(t)}$$

- Temperature profile modification (ICRH, LH) \Rightarrow Hybrid dynamics (scaling laws).

Experimental results

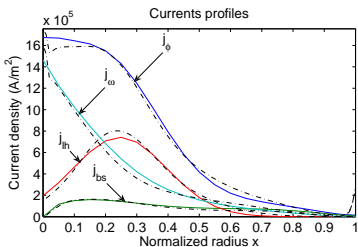
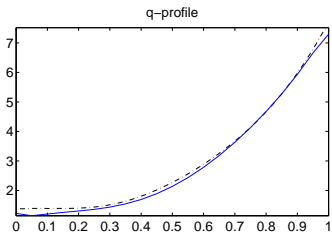
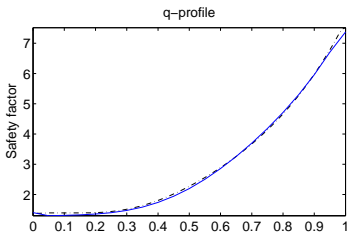
Lower Hybrid effect: shot TS 35109 - variations in $N_{||}$, constant I_p (0.6 MA) and power input (1.8 MW).



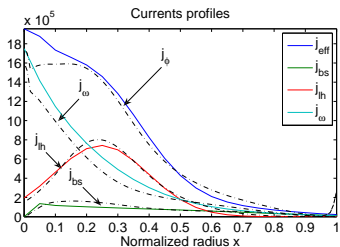
(a) Measured T_e profile

(b) Estimated T_e profile

Figure: ψ_{sim} (—) vs. measurements (---) and CRONOS (— · —): loop voltage (top), $\beta_\theta + I_i/2$ (middle) and edge safety factor (bottom).



(a) Measured T_e profile



(b) Estimated T_e profile

Figure: ψ_{sim} (—) vs. CRONOS (— · —) at $t = 7$ s: safety factor (top) and current densities (effective j_ϕ , LH j_{lh} , ohmic j_ω and bootstrap j_{bs}) profiles (bottom).

NL Model Predictive Control

$$\min_{LH, ECRF} J \doteq \int_0^1 (j_{ref} - (j_{bs} + j_{ohm} + j_{ecrf} + j_{lh}))^2 dx$$

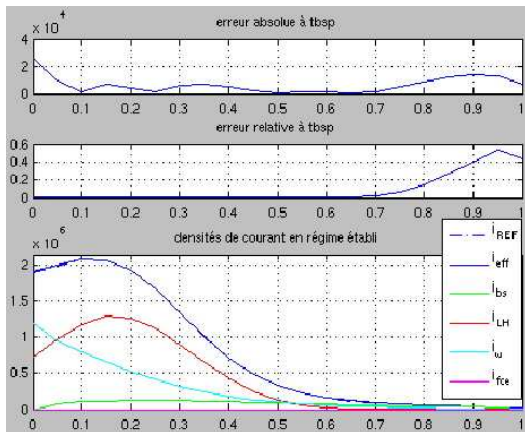


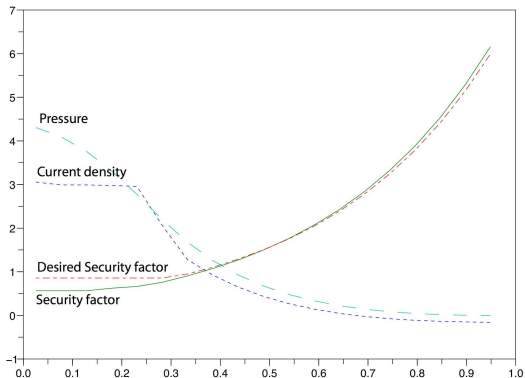
Figure: Error and current profiles with NLMPC

Tracking control

Security factor profile tracking q^*

$$j_{lh} = -j_{bs} \left(\partial_x P_e, \frac{-c_q X}{q^*} \right) - c_j \frac{1}{X} \partial_x \left[X \partial_x \left[\frac{-c_q X}{q^*} \right] \right] + \frac{V}{R\eta}$$

⇒ Easy to implement (P_e & T_e as inputs), high sensibility to q^*



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- Efficient model to represent the main control-relevant profiles: 3 coupled PDE + wave/particule interactions \rightarrow 1 PDE + identified shapes;
- Specific care of the discretization scheme and numerical integration;
- Fast computation ($\approx 1.7s$ for shot TS 33632 with 21 points and $t_s = 10 ms$), suitable for RT NLMPC;
- Variable step spatial discretization \Rightarrow reduce the number of points (i.e. dynamically);
- Robust wrt. measurement noises, discretization steps (spatial and temporal);
- Simplified model and first steps toward output feedback control.

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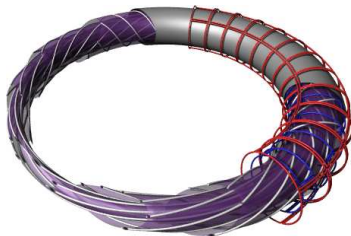
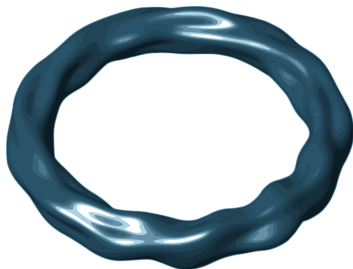
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Stability analysis and model-based control in EXTRAP-T2R with time-delay compensation (Alfvén-lab, KTH, Stockholm)



- Magnetohydrodynamic instabilities (non-symmetric electric currents) = crucial issue for fusion plasmas & ITER
- ⇒ Dedicated experiment: Reverse Field Pinch EXTRAP-T2R, with Intelligent-Shell feedback (4×32 actuator, 4×32 sensor saddle coils)
- MHD control implies to consider
 - the aliasing of spatial harmonics
 - actuation dynamics and control latencies
- ⇒ New model with experimental constraints and model-based control

EXTRAP T2R fusion plasma experiment

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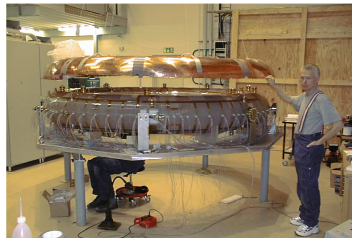
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Machine parameters

- major radius $R_0=1.24$ m
- plasma minor radius $a=18$ cm
- shell time constant $\tau_{ver}=6$ ms
- plasma current $I_p=80$ kA
- electron temp. $T_e=250$ eV
- pulse length $\tau_{pulse} < 60$ ms



EXTRAP T2R vessel and shell during assembly

Pulse lengths $\tau_{pulse} \gg \tau_{ver}$ allow studies of MHD instabilities of the resistive wall mode (RWM) type (growing on the time scale of the shell)

Magnetohydrodynamic unstable modes model

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Resistive-wall mode physics in RFP:

from MHD to perturbed ODE

- Linear stability investigated by periodic spectral decomposition

$$\mathbf{b}(r, t) = \sum_{mn} \mathbf{b}_{mn}(r) e^{j(t\omega + m\theta + n\phi)}$$

Fourier eigenmodes $\mathbf{b}_{mn}(r)$ with growth-rate $\gamma_{mn} = j\omega_{mn}$,

- Ideal MHD modes:

$$\tau_{mn} \dot{\mathbf{b}}_{mn}^r - \tau_{mn} \gamma_{mn} \mathbf{b}_{mn}^r = \mathbf{b}_{mn}^{r, ext}$$

\mathbf{b}_{mn}^r : radial component of perturbed field, $\mathbf{b}_{mn}^{r, ext}$: external active coil, τ_{mn} : penetration time.

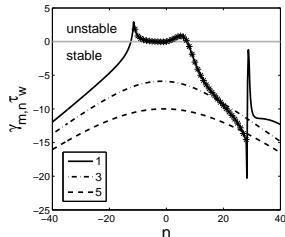


Figure: Growth-rates $\tau_w \gamma_{mn}$. *: Integer- n non-resonant positions (RWMs) for $m = 1$.

MIMO plant modeling by geometric coupling of SISO dynamics

Standard state-space form

- Faraday and Biot-Savart laws + ideal integrator on the sensor coil output voltage (hyp.)

$$\begin{cases} \dot{\mathbf{x}} &= \mathbf{A}\mathbf{x} + \mathbf{B}\mathbf{u} + \mathbf{N}\mathbf{v}_1 & \text{MHD-modes vs. active coil cur. \& exog. signal} \\ \mathbf{z} &= \mathbf{M}\mathbf{x} & \text{optional performance vector} \\ \mathbf{y} &= \mathbf{C}\mathbf{x} + \mathbf{v}_2 & \text{time-integrated sensor voltages \& white noise} \end{cases}$$

- with state matrix elements (instantiated for T2R geometry and routing)

$$A_{mn,m'n'} \sim \gamma_{mn} \delta_{mn,m'n'}$$

$$B_{mn,ij} \sim \tau_{mn}^{-1} \int_{\Omega} e^{-i(m\theta+n\phi)} \left(\hat{\mathbf{r}} \cdot \oint_{I_{ij}} \frac{d\mathbf{l}_{ij} \times (\mathbf{r} - \mathbf{r}_{ij})}{|\mathbf{r} - \mathbf{r}_{ij}|^3} \right) d\Omega$$

$$C_{pq,mn} \sim \int_{\Omega} e^{+i(m\theta+n\phi)} f_{pq} A_{pq} d\Omega$$

Modes coupling and aliasing of spatial frequencies:

- sensors and actuators: single-mode to multiple-mode model affected by *aliasing*;
 - the traditional *IS-regulator* (output convergence) does not counteract the disturbances influence on MHD-modes;
- ⇒ **Precise *mode-control* to address the problem of internal state stability.**

Actuators dynamics, latencies and PID control

- Characteristic times (T2R) → actuation

$$\mathbf{u}_{\text{sys}}(t) \approx \frac{1}{\tau_c s + 1} \frac{K}{\tau_a s + 1} \mathbf{u}_{\text{DAC}}(t - \tau_h)$$

- PID implementation with time-delay

$$\mathbf{u}_{\text{DAC}}(t) = K_p \mathbf{e}(t) + K_i \mathbf{q}(t) + \tau_d^{-1} K_d (\mathbf{e}(t) - \mathbf{e}(t - \tau_d))$$

- ⇒ **Closed-loop dynamics**

$$\dot{\tilde{\mathbf{x}}}(t) = \mathcal{A}_0 \tilde{\mathbf{x}}(t) + \mathcal{A}_1(\theta) \tilde{\mathbf{x}}(t - \tau_h) + \mathcal{A}_2(\theta) \tilde{\mathbf{x}}(t - \tau_h - \tau_d) + \mathcal{E} \mathbf{v}_1(t)$$

Time constants and delays

Symbol	Value/order	Description/comment
$\tau_{w,N}$	13.8 ms	<i>Nominal</i> resistive wall time
τ_w	≈ 10 ms	<i>Experimental</i> resistive wall time
τ_{mn}	$\lesssim \frac{1}{2}\tau_w$	Actual model mode time
τ_{MHD}	$\sim 1\mu s$	Internal MHD activity/fluctuations
τ_d	100 μs	Digital sampling time, controller cycle
τ_h	$\sim 100\mu s$	Control latency, dead time
τ_{CPU}	$< 100\mu s$	Algorithm-dependent part of τ_h
τ_a	8 μs	Active amplifier first-order time
τ_c	1 ms	Active coil L/R -time
$\tau_{A\&D}$	$\sim 1\mu s$	ADC/DAC settle, ns/ μs respectively

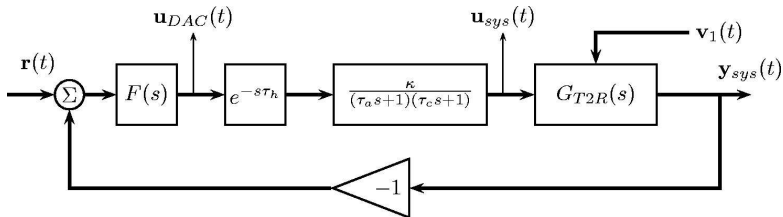
Open-loop error estimation and parameter identification

- Actuators dynamics identification $\rho^{ij} \doteq \{\tau_c^{ij}, \tau_h^{ij}, \kappa^{ij}\}$ from (PRBS).
- Error-field estimation and filtering

$$\begin{cases} \dot{\tilde{\mathbf{x}}} &= \begin{pmatrix} A & N \\ 0 & -\tau_s^{-1}I \end{pmatrix} \tilde{\mathbf{x}} + \begin{pmatrix} B \\ 0 \end{pmatrix} \mathbf{u} + \mathbf{v}'_1 \\ \mathbf{y} &= \begin{pmatrix} C & 0 \end{pmatrix} \tilde{\mathbf{x}} + \mathbf{v}_2 \end{cases}$$

$\tilde{\mathbf{x}}(t) \doteq (\mathbf{x}(t)^T \mathbf{x}_s(t)^T)^T$ estimated by Kalman filter, $\mathbf{x}_s(t)$ is *inter alia* RWM-instabilities

Model summary



Stability analysis and delay effects

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- Infinite spectrum of the Delay Differential Equation

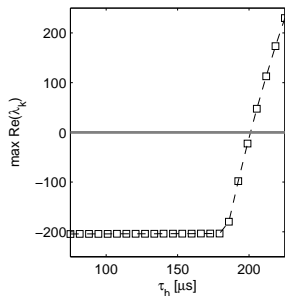
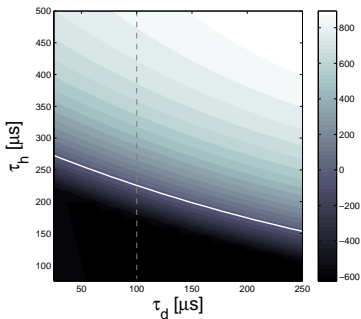
$$\det \Delta(s) = \det \left(sI - \mathcal{A}_0 - \sum_{i=1}^n \mathcal{A}_i e^{-s\tau_i} \right) = 0$$

- Mode-control and perfect decoupling: SISO dynamics (fixed gains)

$$G_{mn}(s) = \frac{1}{\tau_{mn}s - \tau_{mn}\gamma_{mn}} \frac{1}{\tau_c s + 1} \frac{1}{\tau_a s + 1} e^{-s\tau_h}$$

- fictitious but usefull for disturbance rejection and resonant-field amplification analysis

- Spectrum dependence on τ_h : MIMO (rightmost roots) and SISO cases



SISO-set stability in (τ_h, τ_d) -space and MIMO-plant stability w.r.t. τ_h .

⇒ Infinite spectrum model with key periferal (automation) dynamics.

Model-based control and delay compensation

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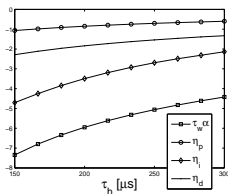
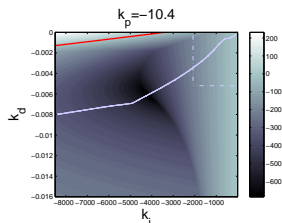
- **Objective:** ensure MHD stability and minimize closed-loop spectral abscissa under PI-TD structure constraints
- **Advantages of TD approach:**
 - varying computational complexity implies varying τ_h
 - free $\tau_h =$ fitting parameter to mimic experimental instability onset

Direct Eigenvalue Optimization

- direct MIMO approach generally nonconvex and nonsmooth
- ⇒ hybrid SISO/MIMO method: minimize the maximum spectral abscissas of the SISO set
- *gradient-sampling* method: *robustified* steepest-descent method suitable for nonsmooth optimization

Optimization results

- Two different parameterizations, implicitly assigning the closed-loop performance and control-input norm:
 - a) varying k_p and searching for the optimal $\tilde{\theta}^* = (k_i^*, k_d^*)$ for a nominal $\tau_h \rightarrow$ Max CL spectrum in (k_i, k_d) -space,
 - b) varying τ_h and determining the full optimal PID $\tilde{\theta}^* = (k_p^*, k_i^*, k_d^*)$
- Robust on the given problems: convergence within 10 – 30 iterations



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- **New T2R experiments:** #20743 – #20755 and #20824 – #20838 - $I_p \approx 85$ kA, shot length $\tau_p \approx 50 - 70$ ms and *reversal* and *pinch* $(F, \Theta) \approx (-0.27, 1.72)$.
- Experiment vs. simulation model: global shape is conserved = suitable for control
- Initial horizontal motion compensated by an external system stronger than IS controller and needs ~ 10 ms to become stationary
- Transient or steady-state generic performance index

$$J_v(\theta) \equiv \frac{1}{t_1 - t_0} \int_{t_0}^{t_1} \sigma_\tau v^T(\tau, \theta) v(\tau, \theta)$$

Performance improvements

Shot#	K_p	K_i	K_d	J_y	J_u	Comment
20743	150	16000	0.05	0.464	1.66	old setting 1
20744	160	16000	0.04	0.509	1.80	old setting 2
20746	106	37500	0.061	0.259	2.12	series a)
20747	126	47500	0.073	0.304	1.94	a)
20827	150	16000	0.05	0.501	1.60	old setting 1
20833	119.6	46800	0.065	0.304	1.77	b)
20835	106.8	39860	0.058	0.288	1.64	b)

⇒ Optimized controllers = 44% reduction of average field energy at the sensors during steady-state period, at the expense of higher input power (+28%).

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- New model for MHD instabilities in T2R, explicitly including important geometrical and engineering aspects
 - Direct closed-loop PID gain optimization for the corresponding DDE model
- ⇒ experimental intelligent-shell feedback in a RFP fusion research device
- Applicability of the model to real experimental conditions
 - Strongly encourage future work, theoretically and experimentally, in both physical modeling and multivariable control for MHD.

General conclusions

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- Thermonuclear fusion motivates key advances in many fields of research.
- Advanced control methodologies definitely make a difference.
- Most problems are associated with transport phenomena and the system complexity.
- Need to “cross the bridge” between physicists and control engineers.
- Controlled fusion aims to become a national field of expertise with ITER.

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